



## Erratum

## Erratum to “Safe radioisotope thermoelectric generators and heat sources for space applications” [Journal of Nuclear Materials 377(2008) 506–521]

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On page 508, the third paragraph of Section 2 should read:

“Despite its high thermal power density (see Table 1), the use of <sup>90</sup>Sr as a direct substitute for <sup>238</sup>Pu in the GPHS architecture and use within a low mass heat source is impractical without enhancement of shielding structures due to Bremsstrahlung and the emission of secondary gamma rays; 2.186 MeV gamma rays from <sup>90</sup>Y [19], the short-lived daughter nuclei from the beta decay of <sup>90</sup>Sr.”

On page 509, right-hand column, the sentence starting in line 7 should read:

“Eight centimetres of aluminium shielding would reduce the intensity of the gamma ray radiation to 10% of its original, compared to the same degree of shielding achieved by 0.38 millimetres of tungsten. In addition to this, <sup>241</sup>Am is also easily extracted from stockpiles of transuranic waste; the bi-product of nuclear fission reactors.”

On page 509, in Table 1, for <sup>90</sup>Sr, in the last row, the following values are corrected:

Theoretical specific activity (Ci g<sup>-1</sup>): 118.87.

Theoretical specific thermal power (W g<sup>-1</sup>): 0.785.

Gamma ray energy (keV): 2186.2 (<sup>90</sup>Y).

Mass required for 5We (g): 127\*.

Due to this change, Figs. 11 and 12 are changed. See new versions enclosed.

On page 515, left-hand column, the second sentence of the last paragraph should read:

“As indicated by the results in Table 2, the received neutron dose rate at a distance of 1 m from the centre of the current <sup>238</sup>Pu fuelled GPHS module is of the order of 0.05816 mSv/h (5.816 mRem/h).”

Following the above corrections and to correct other typos, Table 2 is changed. See new version enclosed.

On page 519, the heading of Fig. 16 should read:

Comparison of neutron dose rates at 1 m radius from oxygen enriched <sup>238</sup>PuO<sub>2</sub> & <sup>241</sup>AmO<sub>2</sub> sources. For approximately equal dose rates, <sup>241</sup>AmO<sub>2</sub> sources have no shielding and <sup>238</sup>PuO<sub>2</sub> sources have 15 cm of Boron loaded Polyethylene neutron shielding.

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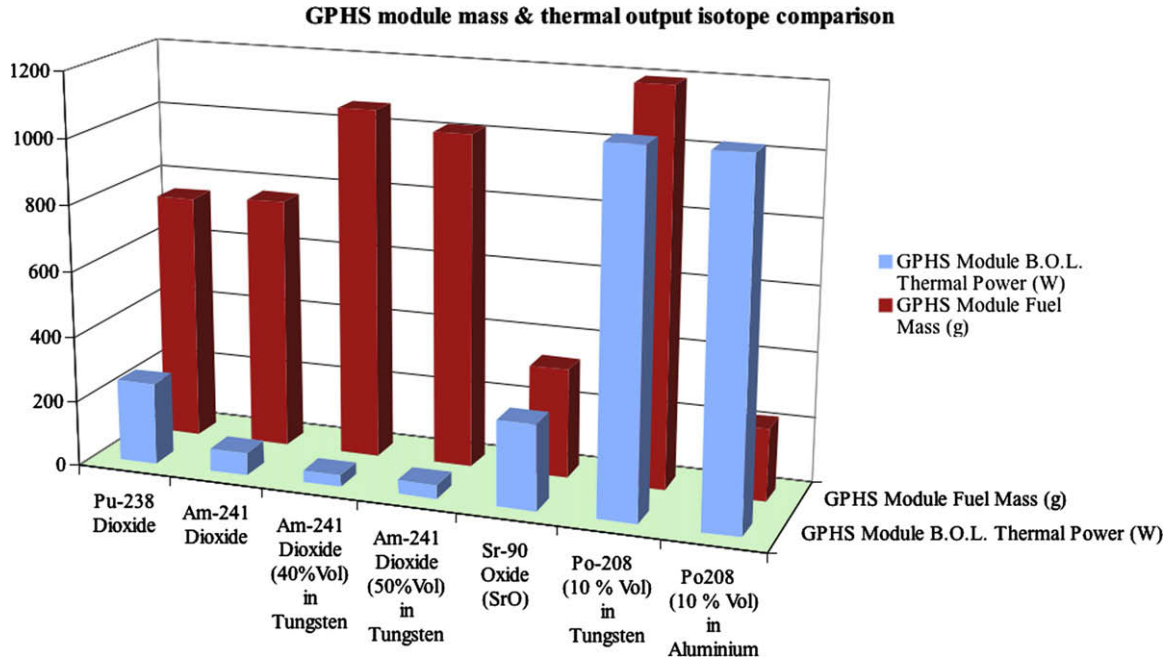


Fig. 11. A thermal performance and fuel mass comparison of substitute isotope candidates (excluding  $^{210}\text{Po}$ ) when used to fuel a GPHS module in contrast to the current performances of the GPHS modules fuelled by  $^{238}\text{PuO}_2$ .

Comparison of neutron dose rates at 1m radius from oxygen-enriched  $^{238}\text{PuO}_2$  &  $^{241}\text{AmO}_2$  sources. For approximately equal dose rates,  $^{241}\text{AmO}_2$  sources have no shielding and  $^{238}\text{PuO}_2$  sources have 15 cm of Boron loaded Polyethylene neutron shielding.

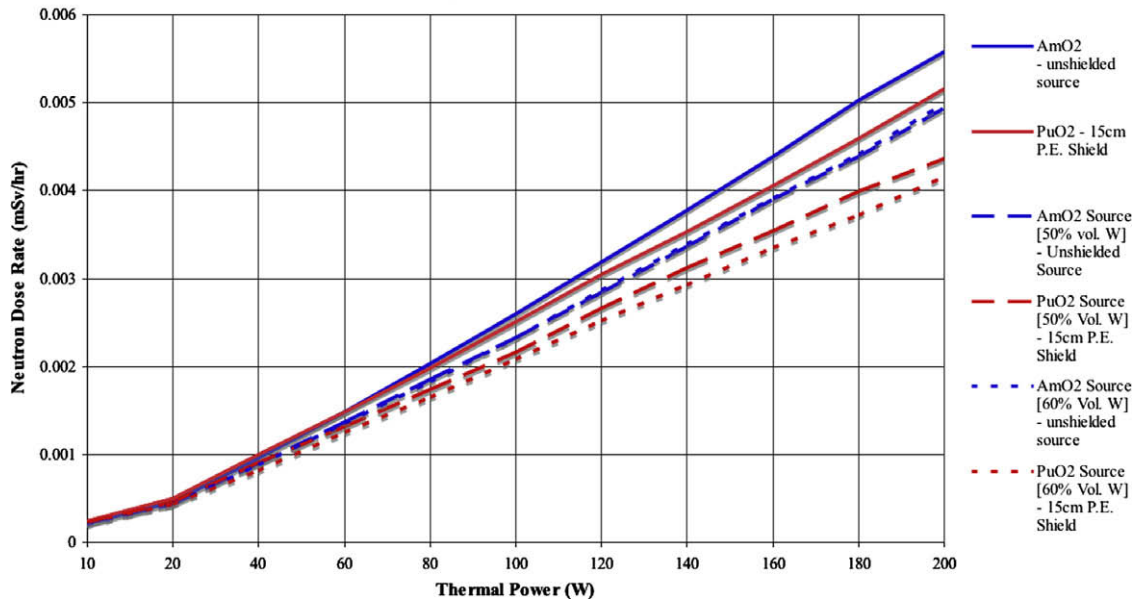


Fig. 12. A thermal performance and fuel mass comparison of all substitute isotope candidates when used to fuel a GPHS module in contrast to the performances of the GPHS modules fuelled  $^{238}\text{PuO}_2$ .

**Table 2**

Alternative isotope study results for the replacement of the current PuO<sub>2</sub> fuel in the GPHS module. Data for oxygen enriched oxides and oxides of natural oxygen composition is given for <sup>238</sup>PuO<sub>2</sub> and <sup>241</sup>AmO<sub>2</sub> fuelled modules. '>' Signifies that a practically unlimited annual exposure time is permissible.

GPHS Module fuel form:	Pu-238 <sup>238</sup> PuO <sub>2</sub> Ceramic pellet	Am-241 <sup>241</sup> AmO <sub>2</sub> (40%vol.) and W (60% vol.) Cermet	Am-241 <sup>241</sup> AmO <sub>2</sub> (50%vol.) and W (50%vol.) Cermet	Am-241 <sup>241</sup> AmO <sub>2</sub> Ceramic pellet	Sr-90 <sup>90</sup> SrO Ceramic pellet	Po-208 <sup>208</sup> Po (10%vol.) and W (90%vol.) Cermet	Po-208 <sup>208</sup> Po (10%vol.) and Al (90%vol.)Cermet	Po-210 <sup>210</sup> Po (10%vol.) and W (90%vol.) Cermet	Po-210 <sup>210</sup> Po (10%vol.) and Al (90%vol.) Cermet
Pellet Mass (g):	$1.88 \times 10^2$	$2.66 \times 10^2$	$2.54 \times 10^2$	$1.91 \times 10^2$	$8.3 \times 10^1$	$2.99 \times 10^2$	$5.50 \times 10^1$	$2.99 \times 10^2$	$5.50 \times 10^2$
Total Fuel Mass (g):	$7.51 \times 10^2$	$1.07 \times 10^3$	$1.02 \times 10^3$	$7.66 \times 10^2$	$3.34 \times 10^2$	$1.20 \times 10^3$	$2.20 \times 10^2$	$1.20 \times 10^3$	$2.20 \times 10^2$
Change in Module Mass (g):	0	+314	+264	+14	-417	+445	-531	+446	-531
Pellet Activity (Ci):	$2.84 \times 10^3$	$2.89 \times 10^2$	$3.47 \times 10^2$	$5.21 \times 10^2$	$9.87 \times 10^3$	$9.00 \times 10^3$	$9.00 \times 10^3$	$6.85 \times 10^4$	$6.85 \times 10^4$
Change in Pellet Activity (Ci):	0	-2.55 × 10 <sup>3</sup>	-2.49 × 10 <sup>3</sup>	-2.32 × 10 <sup>3</sup>	+7.03 × 10 <sup>3</sup>	+6.16 × 10 <sup>3</sup>	+6.16 × 10 <sup>3</sup>	+6.80 × 10 <sup>4</sup>	+6.80 × 10 <sup>4</sup>
Module Activity (Ci):	$1.13 \times 10^4$	$1.16 \times 10^3$	$1.39 \times 10^3$	$2.08 \times 10^3$	$3.95 \times 10^4$	$3.60 \times 10^4$	$3.60 \times 10^4$	$+2.74 \times 10^5$	$+2.74 \times 10^5$
Change in Module Activity (Ci):	0	-1.01 × 10 <sup>4</sup>	-9.91 × 10 <sup>3</sup>	-9.22 × 10 <sup>3</sup>	+2.82 × 10 <sup>4</sup>	+2.46 × 10 <sup>4</sup>	+2.46 × 10 <sup>4</sup>	2.72 × 10 <sup>5</sup>	2.72 × 10 <sup>5</sup>
Module Thermal Power (W):	$2.50 \times 10^2$	$3.63 \times 10^1$	$4.30 \times 10^1$	$6.80 \times 10^1$	$2.61 \times 10^2$	$1.09 \times 10^3$	$1.09 \times 10^3$	$8.36 \times 10^3$	$8.36 \times 10^3$
Change in Module Thermal Power (W):	0	-2.14 × 10 <sup>2</sup>	-2.07 × 10 <sup>2</sup>	-1.93 × 10 <sup>2</sup>	+1.10 × 10 <sup>1</sup>	+8.36 × 10 <sup>3</sup>	+8.36 × 10 <sup>2</sup>	+8.33 × 10 <sup>3</sup>	+8.33 × 10 <sup>3</sup>
Neutron Dose rate at 1 m radius from GPHS (Sv/hr):	$6.534 \times 10^{-4}$ $\pm 9.00 \times 10^{-6}$	$3.036 \times 10^{-5}$ $\pm 7.94 \times 10^{-7}$	$3.788 \times 10^{-5}$ $\pm 2.68 \times 10^{-7}$	$8.238 \times 10^{-5}$ $\pm 1.86 \times 10^{-7}$	-	-	-	-	-
Neutron dose rate as a Percentage of <sup>238</sup> PuO <sub>2</sub> fuelled GPHS dose rate (%):	100	4.6	5.8	12.6	-	-	-	-	-
Gamma dose rate at 1 m radius from GPHS (Sv/hr):	$1.880 \times 10^{-6}$ $\pm 1.09 \times 10^{-8}$	$6.882 \times 10^{-8}$ $\pm 3.17 \times 10^{-10}$	$8.273 \times 10^{-8}$ $\pm 2.79 \times 10^{-10}$	$1.740 \times 10^{-7}$ $\pm 1.44 \times 10^{-9}$	$6.448 \times 10^{-2}$ $\pm 1.90 \times 10^{-4}$	-	-	-	-
Gamma dose rate as a Percentage of <sup>238</sup> PuO <sub>2</sub> fuelled GPHS dose rate (%):	100	3.7	4.4	9.3	$3.854 \times 10^7$	-	-	-	-
Worker maximum recommended exposure time (hours per year):	31	657	527	242	0.31	>	>	>	>
Neutron Dose rate at 1 m radius from GPHS fuelled with oxygen enriched materials (Sv/hr):	$5.816 \times 10^{-5}$ $\pm 1.14 \times 10^{-6}$	$6.162 \times 10^{-7}$ $\pm 1.61 \times 10^{-8}$	$7.691 \times 10^{-7}$ $\pm 5.44 \times 10^{-9}$	$1.672 \times 10^{-6}$ $\pm 3.77 \times 10^{-9}$	-	-	-	-	-
Neutron dose rates for modules fuelled by oxygen enriched materials as a percentage of the <sup>238</sup> PuO <sub>2</sub> fuelled GPHS module dose rate (%):	100	1	1.3	3	-	-	-	-	-
Gamma Dose rate at 1 m radius from GPHS module fuelled with oxygen enriched materials (Sv/hr):	$1.673 \times 10^{-7}$ $\pm 1.38 \times 10^{-9}$	$1.397 \times 10^{-9}$ $\pm 6.44 \times 10^{-12}$	$1.679 \times 10^{-9}$ $\pm 5.66 \times 10^{-12}$	$3.532 \times 10^{-9}$ $\pm 2.91 \times 10^{-11}$	$6.448 \times 10^{-2}$ $\pm 1.90 \times 10^{-4}$	-	-	-	-
Gamma dose rates for modules fuelled by oxygen enriched materials as a percentage of the oxygen enriched <sup>238</sup> PuO <sub>2</sub> fuelled GPHS module dose rate (%):	100	0.8	1	2	$3.854 \times 10^7$	-	-	-	-
Worker maximum recommended exposure time to modules fuelled by oxygen enriched materials (hours per year):	343	$3.24 \times 10^4$	$2.59 \times 10^4$	$1.19 \times 10^4$	0.31	>	>	>	>